D.3 THREE MILE ISLAND

Arranged By: Phathom Donald Electrical Engineering, '14

- The Three Mile Island (TMI) Nuclear Generating Station is located in Pennsylvania.
- Two separate power plants: TMI-1 and TMI-2, each with a capacity of 1,700 megawatts.
- Jointly owned by Pennsylvania Electric Company, Jersey Central Power & Light Company, and Metropolitan Edison Company (Met Ed).



- At TMI-2, the reactor core holds approximately 100 tons of uranium, and contains 36,816 fuel rods.
- The reactor has 69 control rods used by operators to control how much power a plant produces.
- Control rods are held up by magnetic clamps.
- In an emergency, the magnetic field is broken and the control rods drop into the core to halt fission. This is called "scram."

- There is a potential for the release of radioactive materials produced in the reactor core as the result of fission.
- Three Basic Safety Barriers:
 - The fuel rods.
 - The reactor vessel and the closed reactor coolant system loop.
 - The containment building.

- Saturation" the temperature and pressure combination at which water boils and turns to steam.
- Problems from an uncovered core:
 - The temperature may rise to a point where a reaction of water and cladding could begin to damage the fuel rods and produce hydrogen.
 - The temperature may rise above the melting point of the uranium fuel.

ESSENTIAL ELEMENTS

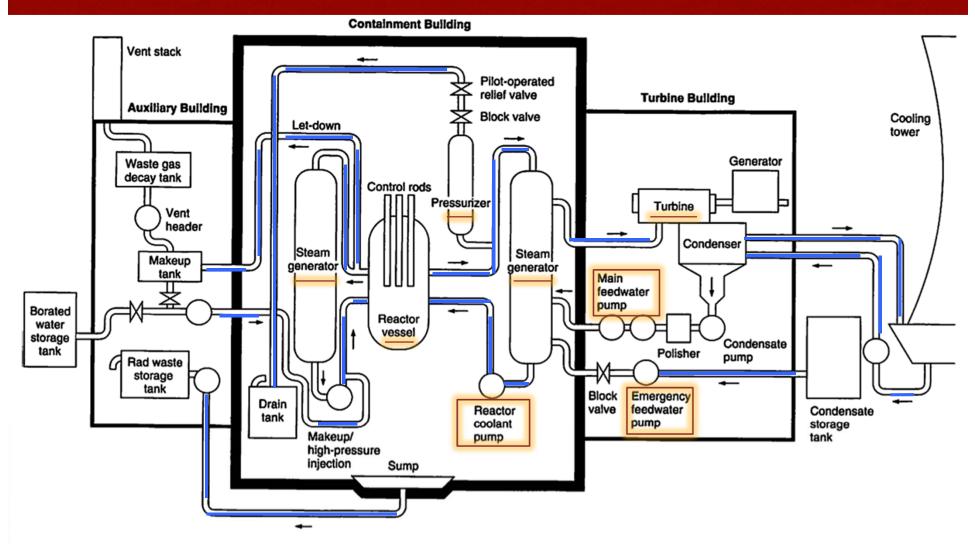


FIGURE D.2

The Three Mile Island nuclear power plant. (Source: John G. Kemeny. *Report of the President's Commission on the Accident at Three Mile Island*, U.S. Government Accounting Office, Washington, D.C., 1979.)

REACTOR COOLANT SYSTEM

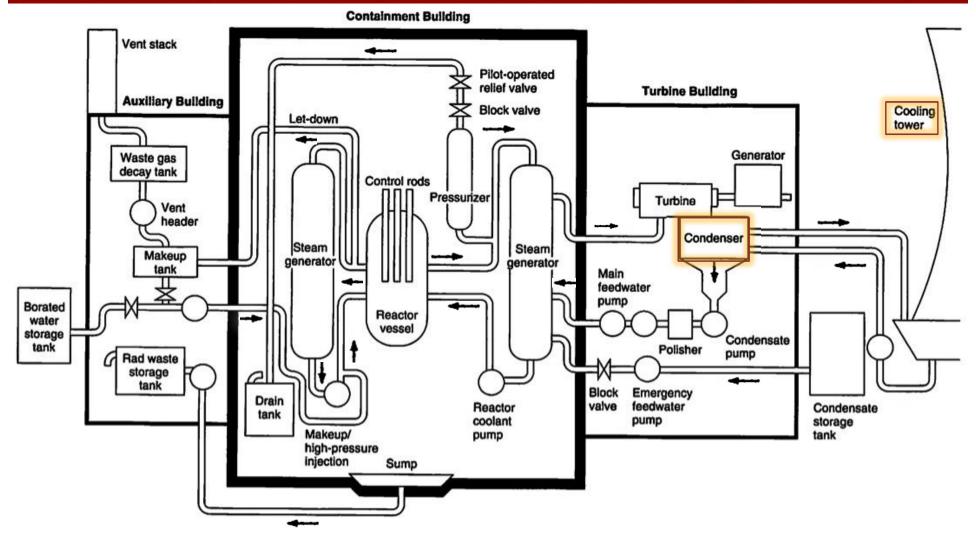
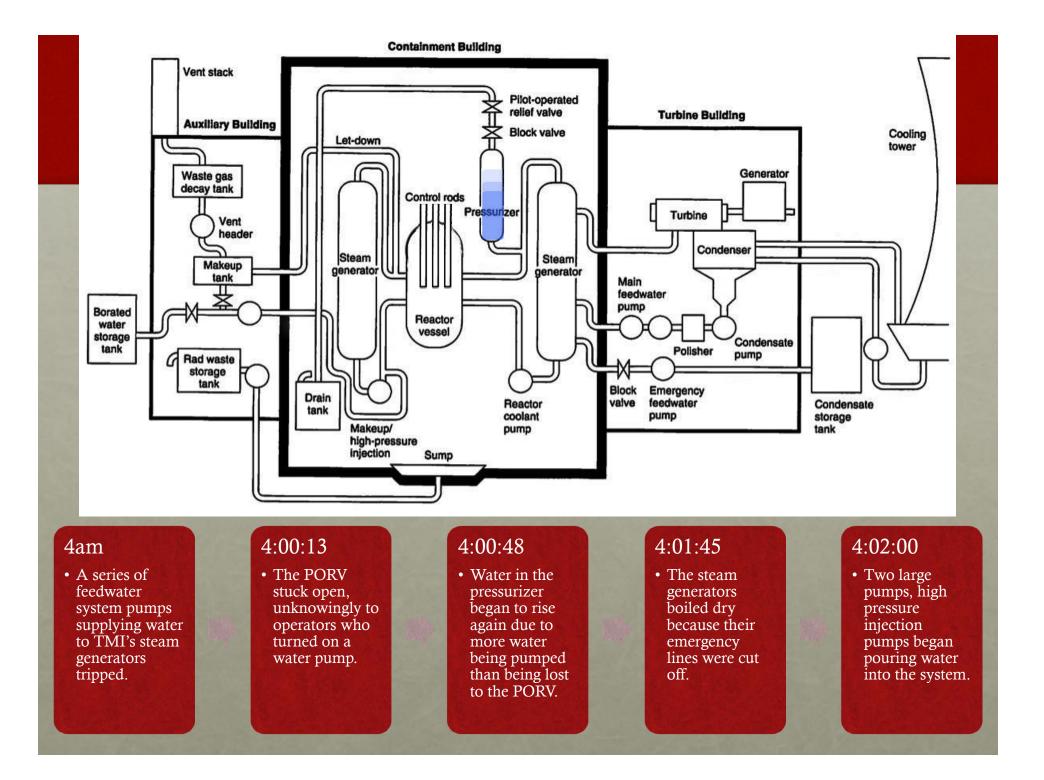
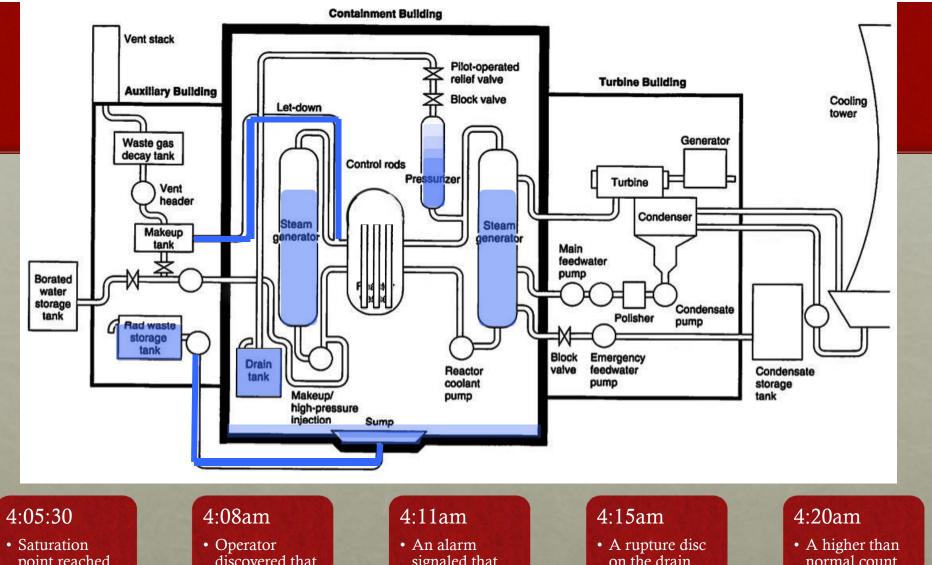


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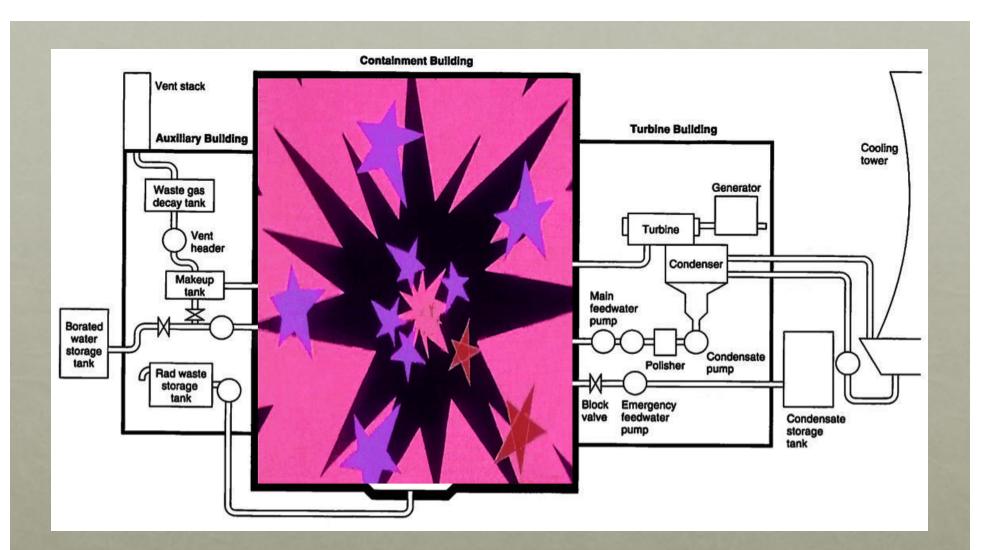




Saturation point reached. Steam bubbles formed in coolant system, displacing coolant water in the reactor. Operator discovered that no emergency feedwater was reaching the generators. An alarm signaled that there was high water in the containment building's sump. • A rupture disc on the drain tank burst as pressure rose. This sent more slightly radioactive water onto the floor. A higher than normal count of neutrons inside the core indicated the presence of steam bubbles in the core.

EVENTS

4:39am • Both sump pumps were stopped but about 8,000 gallons of slightly radioactive water was already pumped.	5:00am • The four reactor coolant pumps began vibrating severely as a result of pumping steam as well as water.	5:14am • Operators shut down two of the pumps.	5:41am • They shut down other two remaining pumps stopping the forced flow of cooling water through the core.	6:00am • Radiation alarms inside the containment building signaled that fuel rod claddings had ruptured.
6:22am • The open block valve was closed, and the loss of coolant was stopped and pressure began to rise.	6:54am • Operators turned on one of the reactor coolant pumps, but it shut down 19 minutes later due to high vibrations.	8:00am • The containment building isolated: a procedure to help prevent radioactive material from escaping into the environment.	8:26am • Operators once again turned on the emergency core cooling system's high pressure injection pump.	10:30am • Pumps finally cover the core again fully.



At 1:50pm on Wednesday, a "thud" was heard in the control room. It was the sound of a hydrogen explosion inside the containment building. However, it was not recognized until late Thursday. The noise was dismissed at the time as a slamming of a ventilation damper.

AFTERMATH OF ACCIDENT

- Cost of accident, including cleanup of the buildings and disposal of approximately 1 million gallons of radioactive water, a substantial amount of radioactive gases, and solid radioactive debris: \$1 billion to \$1.86 billion.
- There was very little radioactive material released outside the plant.
- There was a lowering of public confidence in the industry and its regulatory agencies.

CAUSAL FACTORS



Level 2

Conditions That Allow Those Events

Level 3

Constraints and Conditions That Account for the Level 2 Conditions

LEVEL 2 CONDITIONS

- Containment building remained intact, despite the explosion.
- Their training gave insufficient emphasis to a fundamental understanding of the reactor and to the principles of reactor safety.
- Inadequate written operating and emergency procedures in use at TMI. They contained many errors and imprecise terminology.
- TMI-2 had repeated problems with the condensate polishers. These polishers probably initiated the accident.

LEVEL 2 CONDITIONS

- Control room was not adequately designed with the management of an accident in mind.
- Control room design and information presentation that confused the operators:
 - No way to suppress unimportant signals so that they could concentrate on the important ones
 - Poor arrangement of controls and indicators. Information was not presented in a clear and understandable form.
- John Kemeny of the Kemeny report stated that the technology in the TMI control rooms were "at least twenty years behind in times."



LEVEL 2 CONDITIONS

- Plant procedures were deficient:
 - There was no systematic check required on the status of the plant.
 - Performance of surveillance tests was not adequately verified to be sure that the procedures were followed correctly.
 - After the accident, radiological control practices were observed to be deficient.
- The maintenance force was reduced to save money.
- There were many shutdowns, and a variety of equipment was out of order.
- A number of equipment had a poor maintenance history without adequate corrective action: pressurizer level transmitter, make-up pump switches, and the condensate polishers.

LEVEL 3 CONDITIONS & CONSTRAINTS

- A control room operator complained to his management about problems with the control room, but no corrective action was taken.
- The reactor at TMI has a once-through cooling system, with a small volume of water compared to other U.S. reactors.
- Designers paid little attention to the interaction between humans and machines under the rapidly changing and confusing conditions of an accident.
- The U.S. nuclear industry felt that a major accident could not happen in the United States.
- Supervisors overloaded with excessive paperwork not related to supervision.

LEVEL 3 CONDITIONS & CONSTRAINTS

- NRC requirements at that time were inadequate. They didn't require a quality assurance plan to be applied to the plant as a whole, only "safety-related" systems.
- The operator instructional program did not lead to sufficient understanding of reactor systems.
- Licensing process also concentrated heavily on equipment. But, "safety-related" items didn't need to be reviewed in the licensing process.
- At that time, plants could receive an operating license with several safety issues still unresolved.
- The existence of a state emergency or evacuation plan was not required. The emergency plan did not require the utility to notify state or local authorities in the event of a radiological accident.

THE END